

***JAEA's R&D Activities
on Transmutation Technology
for Long-lived Nuclear Wastes***



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Background (1/3)

- ◆ Sustainable use of nuclear power depends on:
 - **Safety** - **Non-proliferation** - **Radioactive waste** - **Economy**
- ◆ To reduce the burden of radioactive waste management, the Partitioning and Transmutation (P&T) Technology has been actively studied in Japan for more than 30 years.
 - Japan Atomic Energy Agency (JAEA) has been studying this technology since the era of the former institutions; JAERI and PNC/JNC.
 - In Japan, the OMEGA Project was initiated in 1988. Two Check & Reviews (C&Rs) were carried out by the Atomic Energy Commission (AEC) in 2000 and 2009.
 - Since the first C&R, JAEA has been mainly studying P&T of Minor Actinides (MAs), while P&T of Long-Lived Fission Products (LLFPs) was mainly studied by universities.

Background (2/3)

- ◆ After the accident of Fukushima Daiichi Nuclear Power Plant in 2011, public concern to spent fuel management increased in Japan.
- ➔ P&T for long-lived nuclides, therefore, drew the attention from public, media and politicians.
- ◆ In 2014, Japanese Government issued the “Strategic Energy Plan”, where the importance of research and development (R&D) on volume reduction and mitigation of degree of harmfulness of radioactive waste (i. e. P&T) by fast reactor and/or accelerator was described.

Background (3/3)

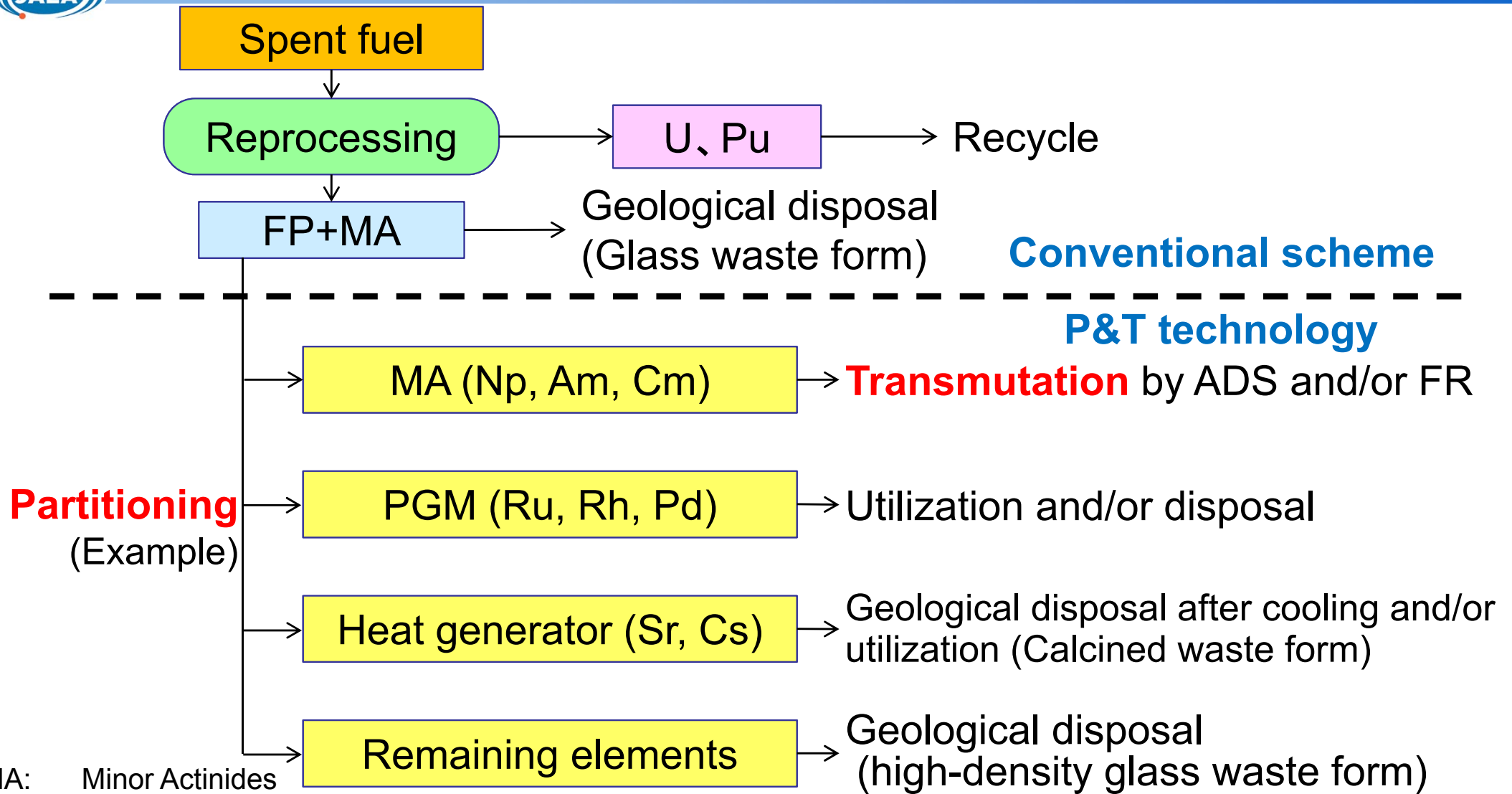
- ◆ Up to now, candidate site for the deep geological repository has not yet been nominated, while the “Map of Scientific Characteristics” was released in July, 2017.
- ◆ In July, 2018, Japanese Government revised the “Strategic Energy Plan”, where promotion of R&D on P&T was mentioned again.

Present Status around P&T in JAEA

- ◆ In December, 2016, Japanese Government decided the shut-down of the prototype fast breeder reactor “MONJU”.
- ◆ JAEA released a middle and long-term plan for facility management, where 44 out of 89 facilities (including JMTR and FCA) will be closed.
- ◆ The Japan Proton Accelerator Research Complex (J-PARC) achieved 1-MW operation (~ 1hr.) in July, 2018. Nevertheless, construction of the Transmutation Experimental Facility (TEF) of J-PARC is still pending.



JAEA's Strategy on P&T



Partitioning
(Example)

Conventional scheme

P&T technology

- MA: Minor Actinides
- FP: Fission Products
- PGM: Platinum Group Metal
- FR: Fast Reactor
- ADS: Accelerator Driven System

Major Long-lived Nuclides in Spent Nuclear Fuel

Actinides

Trans-uranic elements (TRU)

Minor actinides (MA)

Nuclide	Half-life (year)	Dose coefficient (μSv/kBq)	Mass (per 1tHM)
U-235	0.7B	47	10kg
U-238	4.5B	45	930kg

Nuclide	Half-life (year)	Dose coefficient (μSv/kBq)	Mass (per 1tHM)
Pu-238	87.7	230	0.3kg
Pu-239	24k	250	6kg
Pu-240	6.6k	250	3kg
Pu-241	14.3	4.8	1kg

Nuclide	Half-life (year)	Dose coefficient (μSv/kBq)	Mass (per 1tHM)
Np-237	2.14M	110	0.6kg
Am-241	432	200	0.4kg
Am-243	7.4k	200	0.2kg
Cm-244	18.1	120	60g

Fission products (FP)

Nuclide	Half-life (year)	Dose coefficient (μSv/kBq)	Mass (per 1tHM)
Se-79	0.3M	2.9	6g
Sr-90	28.8	28	0.6kg
Zr-93	1.53M	1.1	1kg
Tc-99	0.21M	0.64	1kg
Pd-107	6.5M	0.037	0.3kg
Sn-126	0.1M	4.7	30g
I-129	15.7M	110	0.2kg
Cs-135	2.3M	2.0	0.5kg
Cs-137	30.1	13	1.5kg

The other 30~40kg of FPs are stable or short-lived ones

Dose Coefficient:
 Committed dose (Sv) per unit intake (Bq), indicating the magnitude of influence of radioactivity to human body. α-activity is more influential than β,γ-activity.

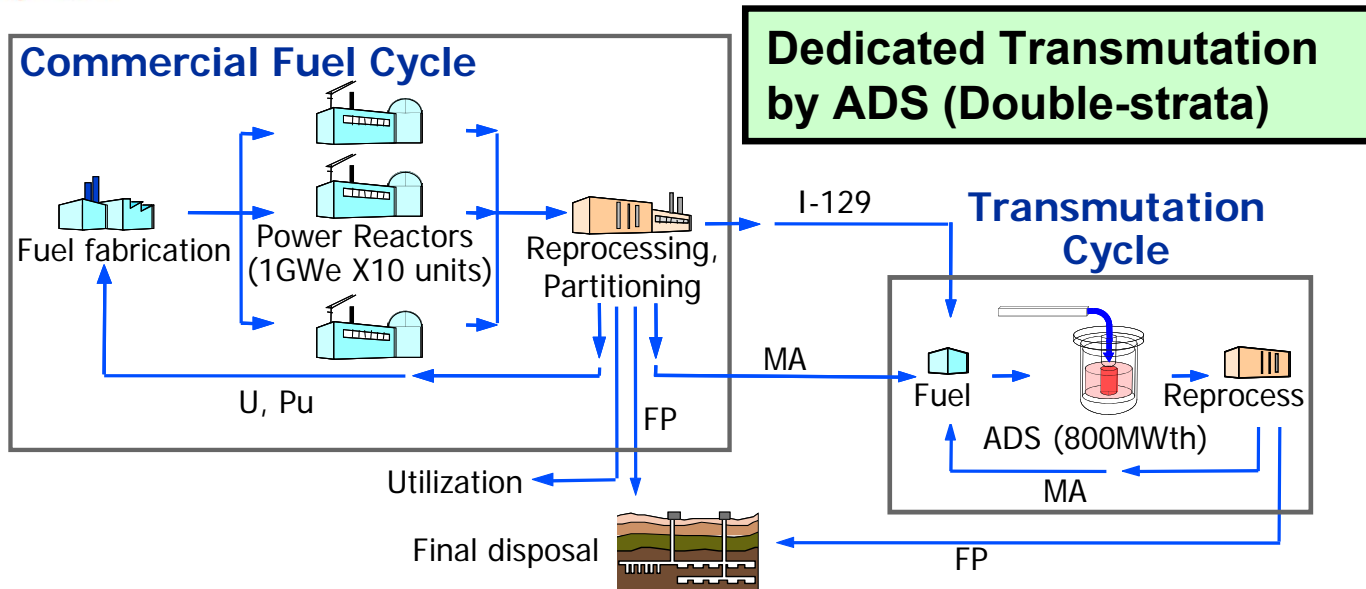
Examples of "Reduced Half-life"

"Reduced Half-life" can be written as $T_{1/2} = \ln 2 / (\varphi \cdot \sigma)$, if there is no competitive reaction nor production reaction, where φ is neutron flux and σ is reaction cross section.

Nuclide	Half-life (year)	Reaction	Neutron energy	Cross section σ (JENDL-4.0) (barn= 10^{-24} cm ²)	Neutron flux φ (/cm ² /s)		
					10 ¹³	10 ¹⁴	10 ¹⁵
					Reduced half-life (year)		
Np-237	2.14M	n,f	Fiss. Spec.	1.336	1,600	160	16
Am-241	432	n,f	Fiss. Spec.	1.378	1,600	160	16
Tc-99	211k	n, γ	Maxwell	23.68	93	9.3	0.93
Sn-126	230k	n, γ	Maxwell	0.09	24,000	2,400	240
I-129	15.7M	n, γ	Maxwell	30.33	72	7.2	0.72
Cs-135	2.3M	n, γ	Maxwell	8.304	260	26	2.6
Cs-137	30.1	n, γ	Maxwell	0.27	8,100	810	81

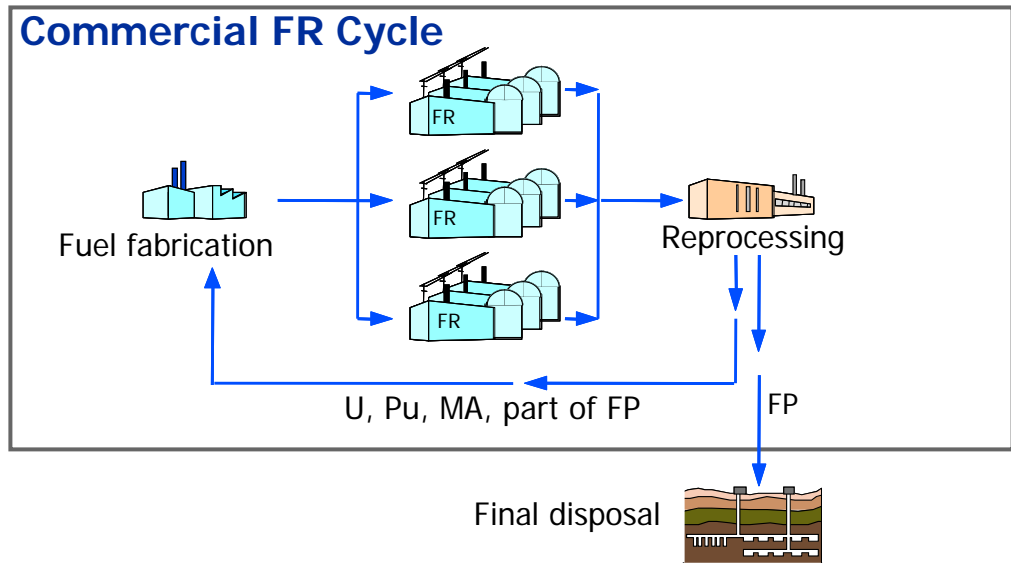
Condition of realistic transmutation: $\varphi \cdot \sigma > 10^{15}$ (barn/cm²/s)

Two Types of Fuel Cycles for P&T



- Transmutation cycle is attached to commercial cycle.
- ADS or ABR is used as dedicated transmutation system.
- MA can be confined into a small cycle and transmuted efficiently.

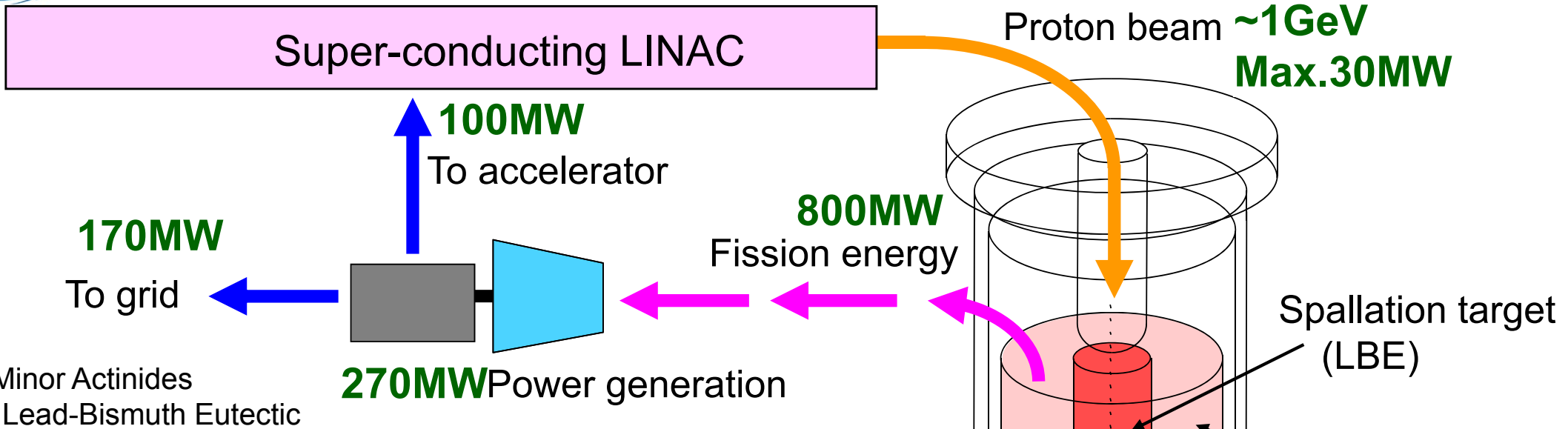
Homogeneous Recycle in FR



- MA is transmuted by commercial FR power plants.
- 5% MA (max.) is added to conventional FR fuel (MOX, Metal, ...)

FP: Fission Product
 MA: Minor Actinide (Np, Am, Cm)
 ADS: Accelerator-Driven System
 ABR: Actinide Burner Reactor
 FR: Fast Reactor

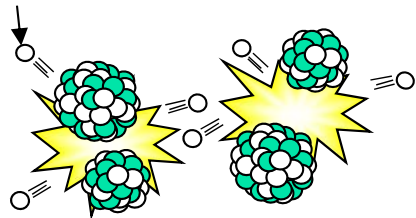
Accelerator Driven System (ADS)



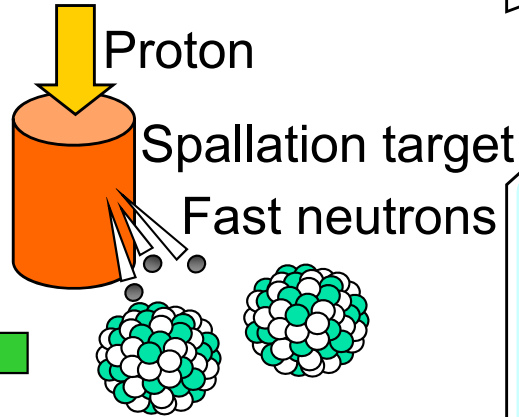
Transmutation by ADS

Utilizing chain reactions in subcritical state

Fission neutrons



Short-lived or stable nuclides

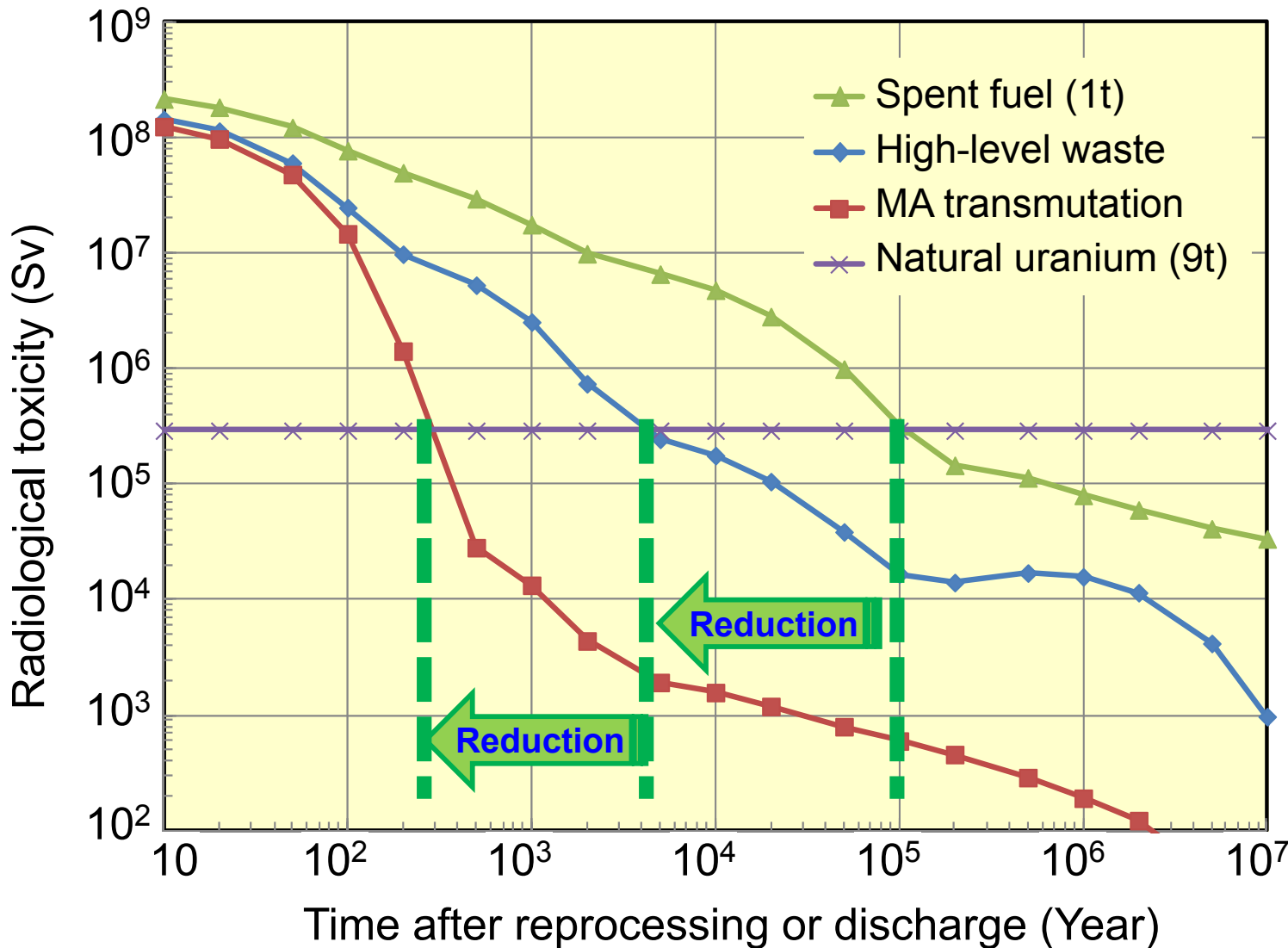


Long-lived nuclides (MA)

Characteristics of ADS:

- Chain reactions stop when the accelerator is turned off.
- LBE is chemically stable.
 - ➔ **High safety can be expected.**
- High MA-bearing fuel can be used.
 - ➔ MA from **10 LWRs** can be transmuted.

Reduction of Radiological Toxicity by P&T



Radiological Toxicity:
Amount of radioactivity weighted by dose coefficient of each nuclide.

- Normalized by 1t of spent fuel.
- 9t of natural uranium (NU) is raw material of 1t of low-enriched uranium including daughter nuclides.

Time period to decay below the NU level:

- Spent fuel **100,000y**
- High-level waste **5,000y**
- MA transmutation **300y**

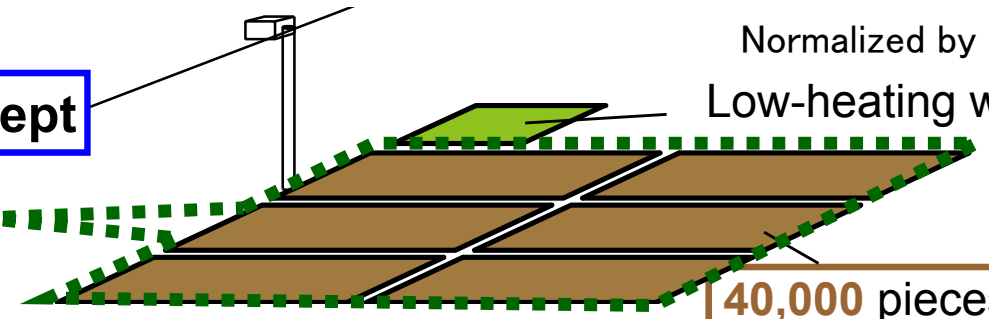
Compact Disposal by Coupling with Long-term Storage



Conventional Concept

1.8 km²

(Vertical emplacement in crystalline rock)



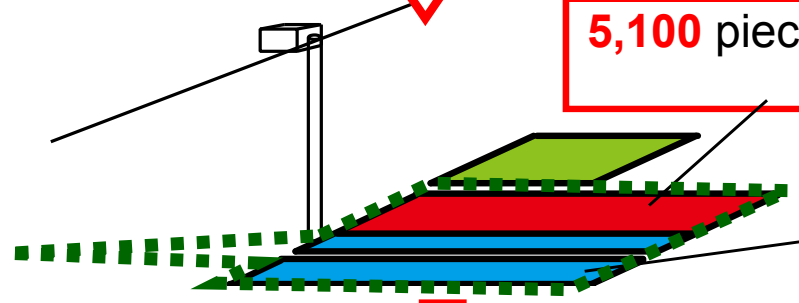
Normalized by 32,000tHM of 45GWd/t spent fuel
 Low-heating wastes (0.13km²)

CT: Cooling time before disposal

40,000 pieces of glass waste forms (1.8km²)
 (CT: **50 y**)

**MA transmutation
 +
 FP Partitioning**

0.41 km²

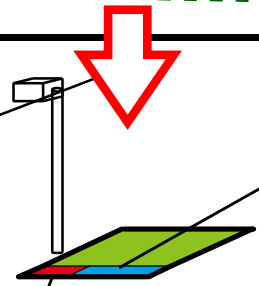


5,100 pieces of Sr-Cs calcined forms (0.23km²)
 (CT: **130 y**)

8,300 pieces of highly-loaded glass waste forms (0.18km²)
 (CT: **5 y**)

**MA transmutation
 +
 FP Partitioning
 +
 Long-term storage of Sr+Cs**

0.015 km²



8,300 pieces of highly-loaded glass waste forms (0.01km²)
 (CT: **45 y**)

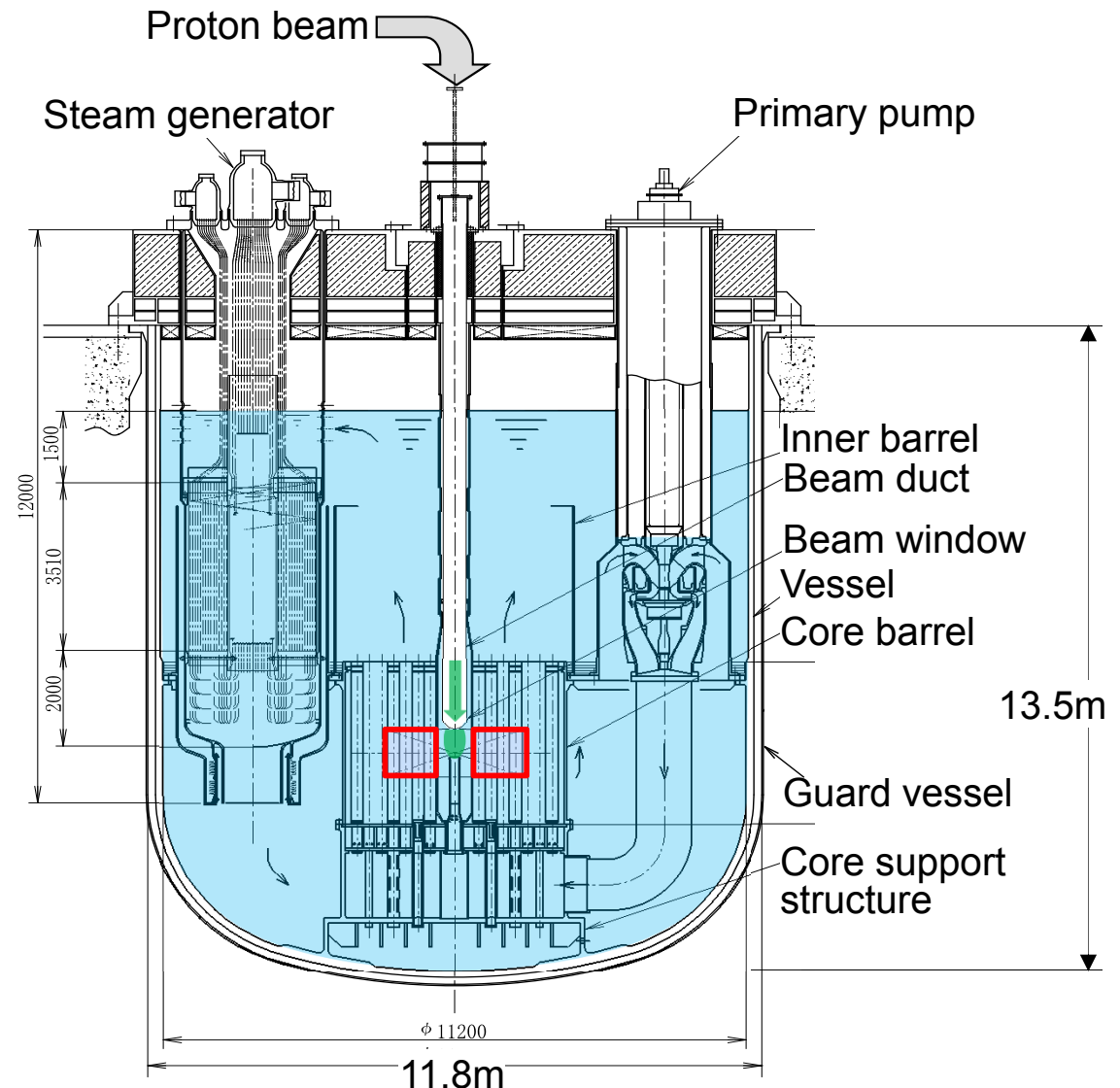
5,100 pieces of Sr-Cs calcined forms (0.005km²)
 (CT: **320 y**)

Ref. : K. Nishihara, et al., "Impact of Partitioning and Transmutation on LWR High-level Waste Disposal", J. Nucl. Sci. Technol., 45(1), 84-97 (2008).

Conceptual Specification and View of ADS



- Proton beam: 1.5GeV
- Spallation target: Pb-Bi
- Coolant: Pb-Bi
Inlet: 300°C, Outlet: 407°C
- Maximum $k_{\text{eff}} = 0.97$
- Thermal output: 800MWt
- MA initial inventory: 2.5t
- Fuel composition:
(MA +Pu)N + ZrN
- Transmutation rate:
10%MA / y
- Fuel exchange:
600EFPD, 1batch
- Primary pump: 2 units
- Steam generator: 4 units
- Decay heat removal system:
3 units



Requirements for Accelerator

High reliability:

- In the case of a saturated stream turbine cycle.
 - Trip longer than 5s may cause thermal stress.
 - Trip longer than 300s may cause turbine stop.

High efficiency:

- In the case of $k_{sub} = 0.95$, 800MWth is achievable by 27MW
- 250MWe can be generated.
- **At least 15%, preferably more than 30% efficiency is necessary to obtain positive energy balance.**

High cost performance:

- Compact design is preferable.

Low beam loss

- To keep accessibility for maintenance, 1W/m loss is maximum.

JAEA's first candidate is Superconducting proton LINAC

- High efficiency in CW mode.
- High magnetic field is achievable.
- Large-size duct is applicable.

Japan Proton Accelerator Research Complex: J-PARC



Jan. 28, 2008



Transmutation Experimental Facility (TEF) of J-PARC

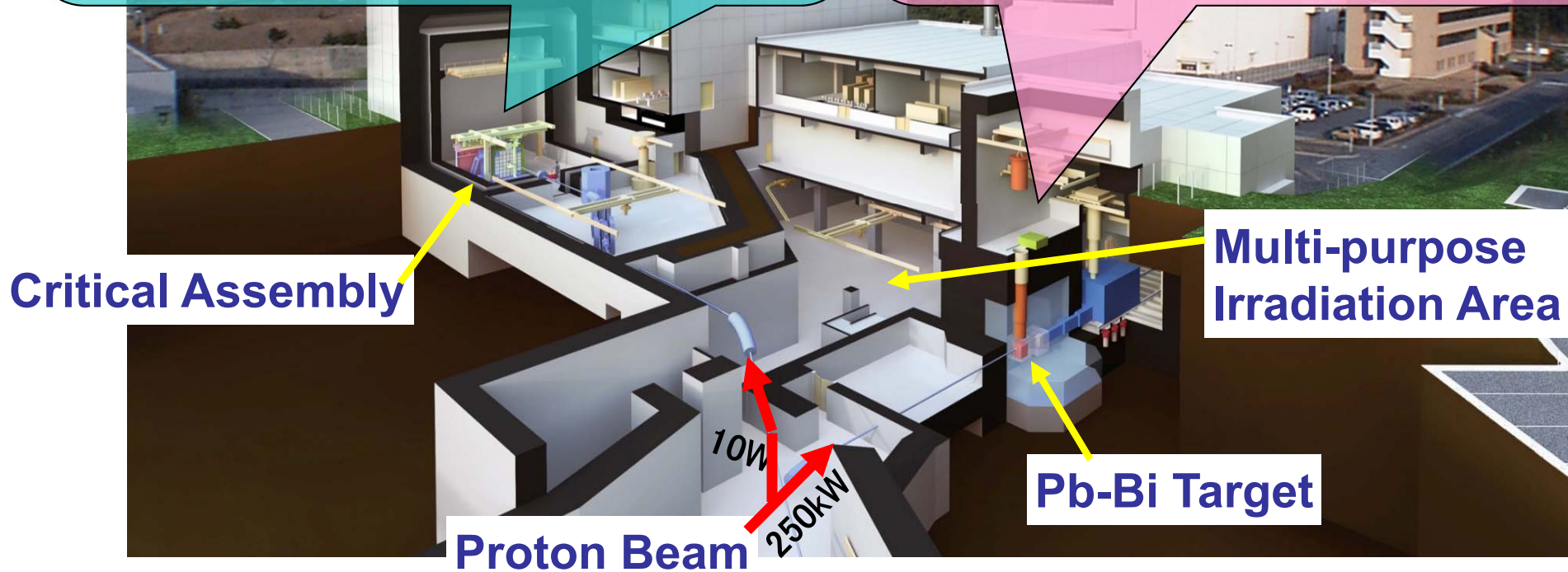


Transmutation Physics Experimental Facility: TEF-P

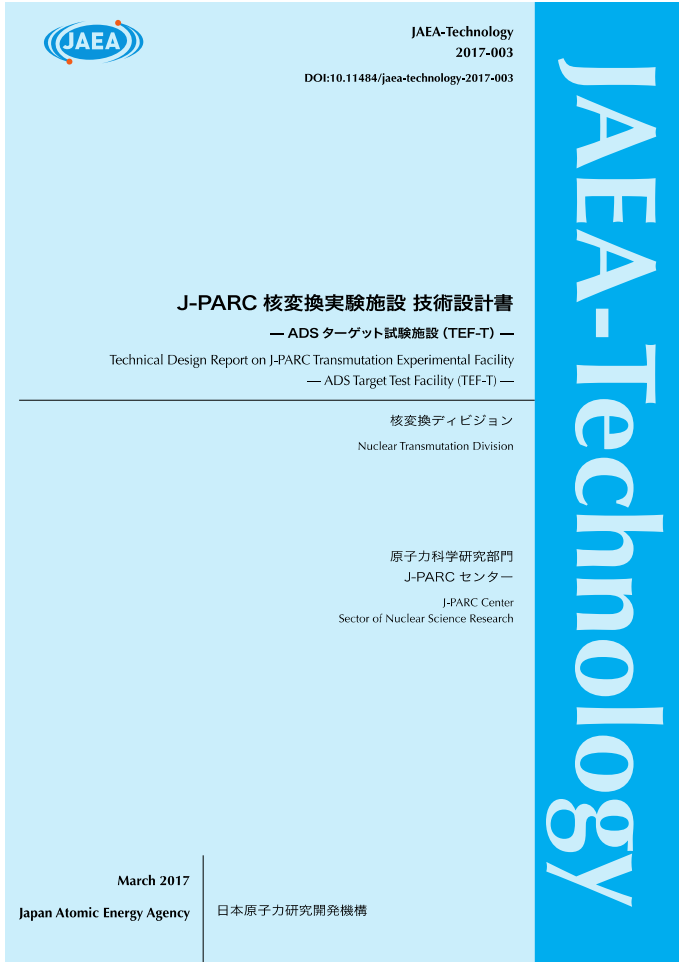
Purpose: To investigate physics properties of subcritical reactor with low power, and to accumulate operation experiences of ADS.
Licensing: Nuclear reactor: (Critical assembly)
Proton beam: 400MeV-10W
Thermal power: <500W

ADS Target Test Facility : TEF-T

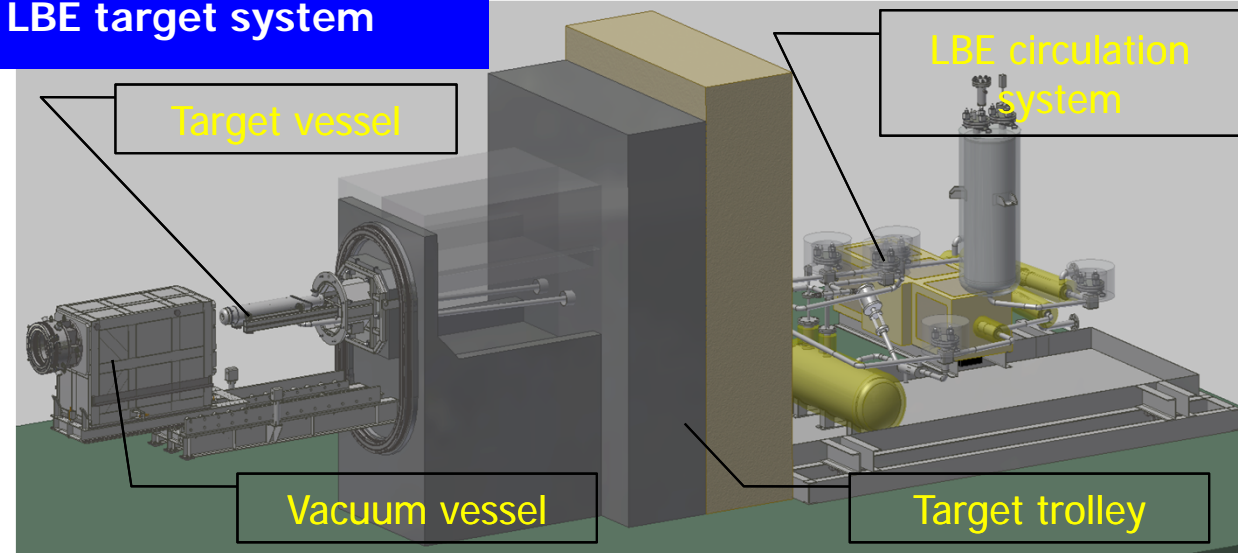
Purpose: To research and develop a spallation target and related materials with high-power proton beam.
Licensing: Particle accelerator
Proton beam: 400MeV-250kW
Target: Lead-Bismuth Eutectic (LBE, Pb-Bi)



ADS Target Test Facility (TEF-T)



LBE target system



- Design of all TEF-T components has been made with considering operation and maintenance procedure.
 - LBE target system
 - Target station
 - PIE facilities
 - Waste management system
 - Proton beam transport
 - Ancillary facilities & building
 - Safety, etc.
- The basic design has been compiled in a report. **JAEA-Technology 2017-003** (sorry, in Japanese)

R&D for TEF-T

■ IMMORTAL

- Purpose: Demonstration of TEF-T target operation
- LBE circulation tests at the max. temp. (500°C), and those with simulating heat input by the proton beam and its removal by operating HX were succeeded.
- System transient exp. and analysis are to be performed.

■ OLLOCHI

- Purpose: Acquirement of materials corrosion data in flowing LBE under static conditions and in-situ loading of mechanical stress
- The loop is under commissioning.

Integrated Multi-functional Mockup for TEF-T Real-scale Target Loop



IMMORTAL

Oxygen-controlled LBE Loop for Corrosion tests in High temperature



OLLOCHI

Oxygen concentration sensors under development



Remote handling cutting and welding of LBE piping for the target vessel replacement



Road map of ADS materials to realize 1st ADS

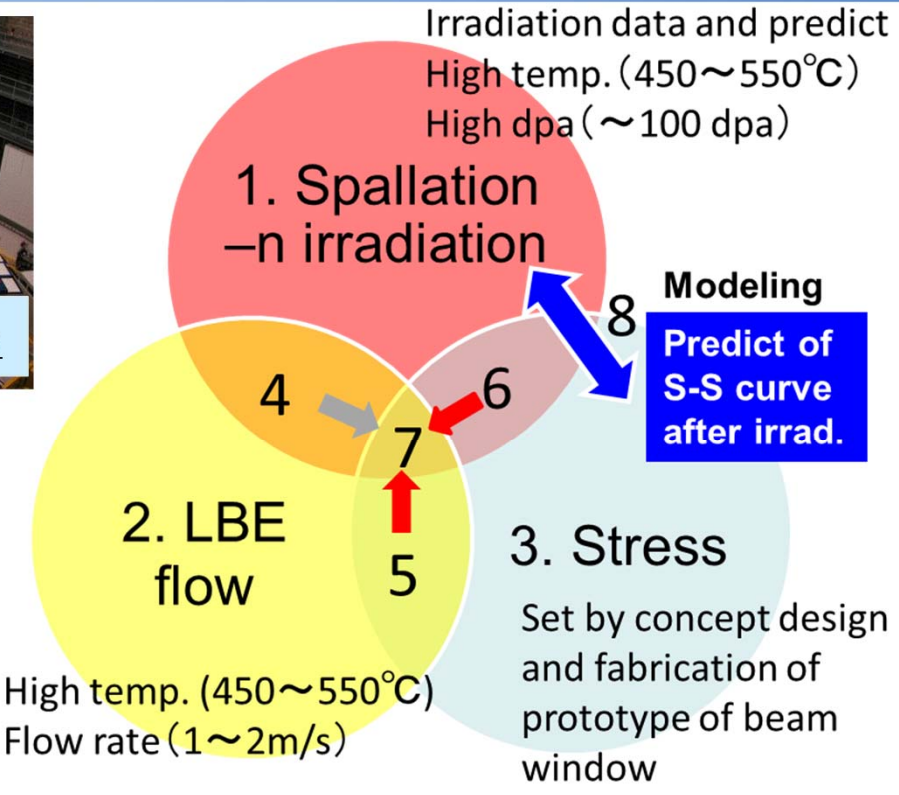
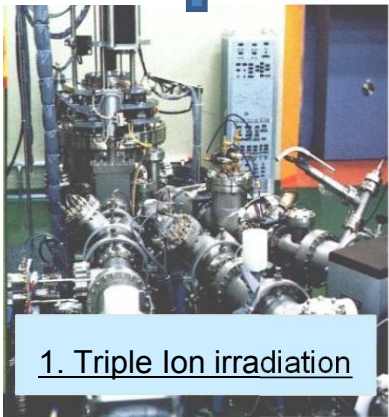
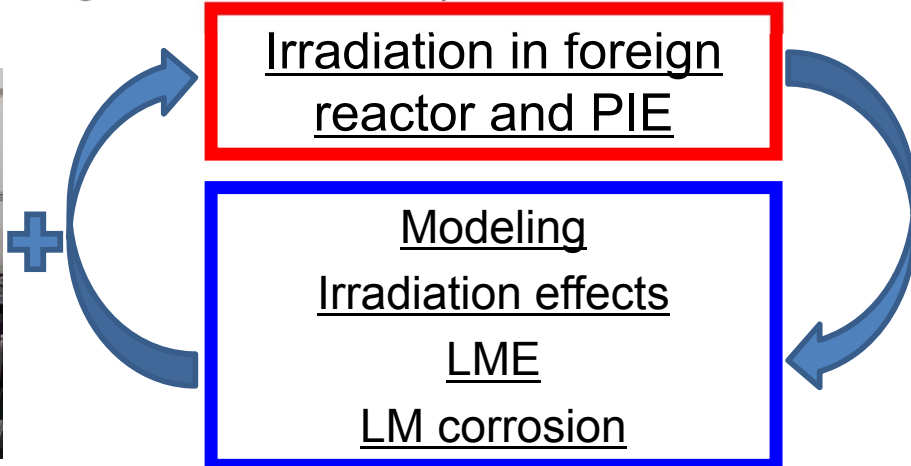


Fig. Three factors exposed for ADS materials



Usage of existing facility and apparatus

1. Spallation-n irradiation (STIP)
Ion irradiation (TIARA, QST)
Neutron irradiation (BR-2, etc.)
2. High temp. corrosion loop
3. Mechanical tests in LBE
5. Corrosion test in LBE flow
6. New PIE facility

4,7 Demonstration facility

·ADS irradiation facility
·MYRRHA
·...

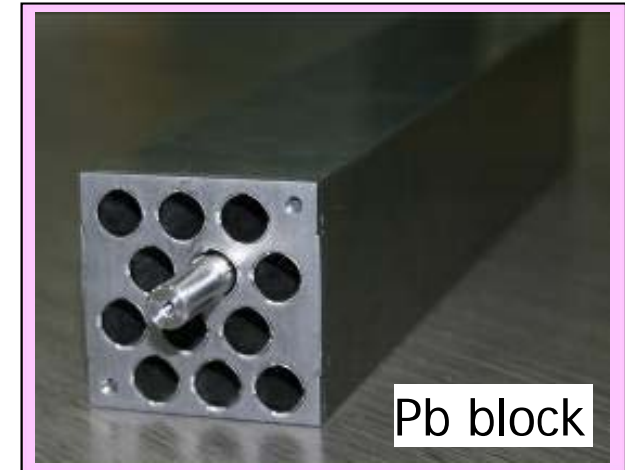
7. 1st ADS



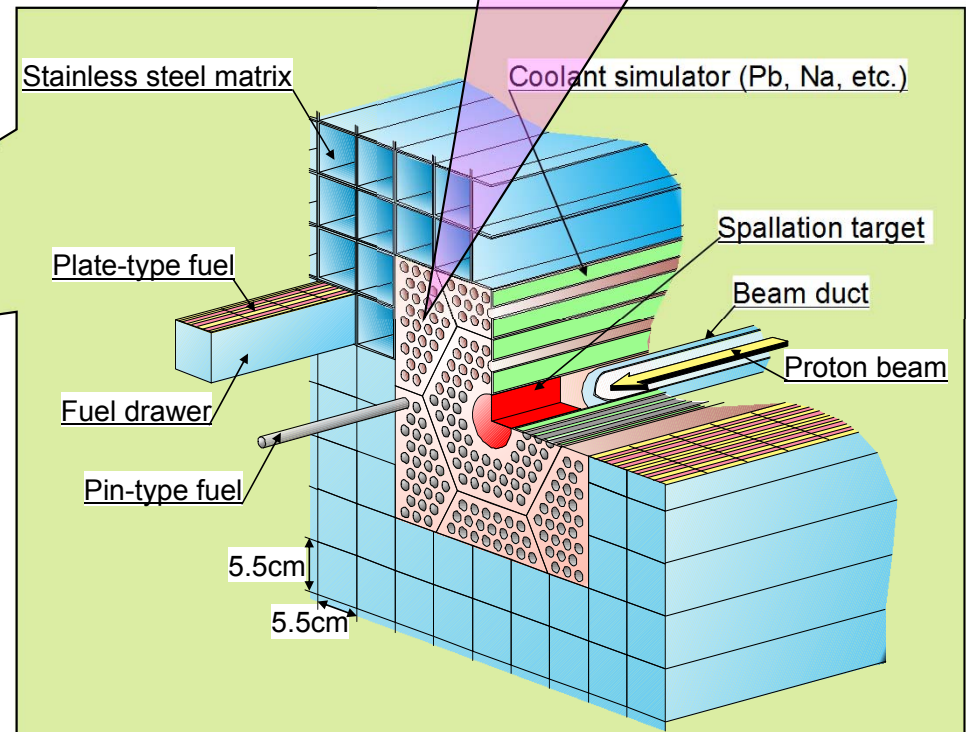
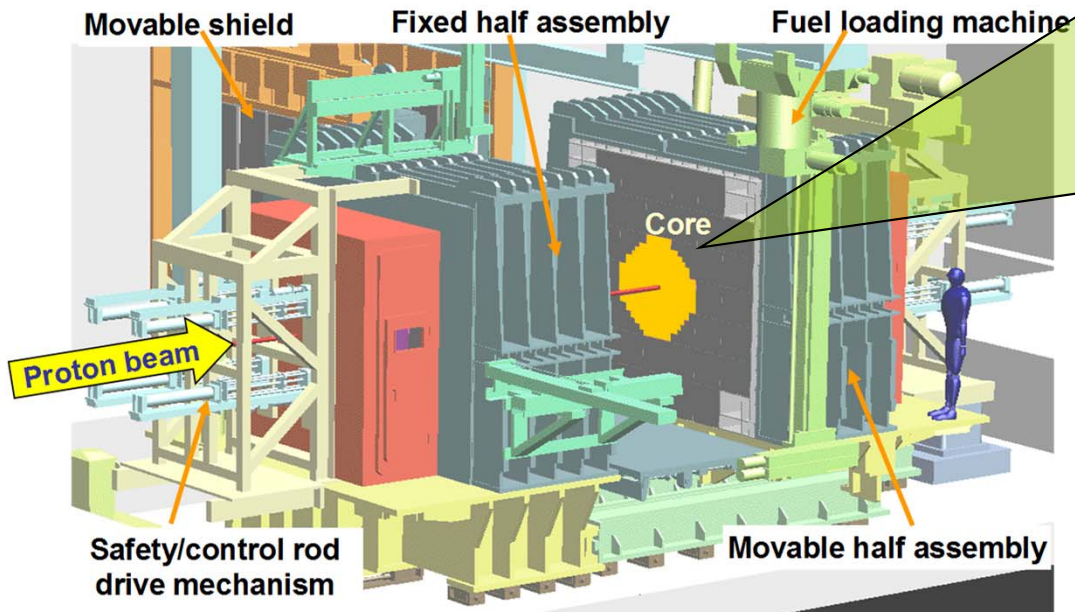
Transmutation Physics Experimental Facility (TEF-P)



- **Critical facility** for reactor physics and nuclear data of transmutation systems: both ADS and FBR.
- Neutron source: $10^{12}n/s$, 25Hz. 1ns pulsed beam is available by laser charge exchange technique.
- **Pin-type MA fuel can be used** with appropriate cooling and remote handling.

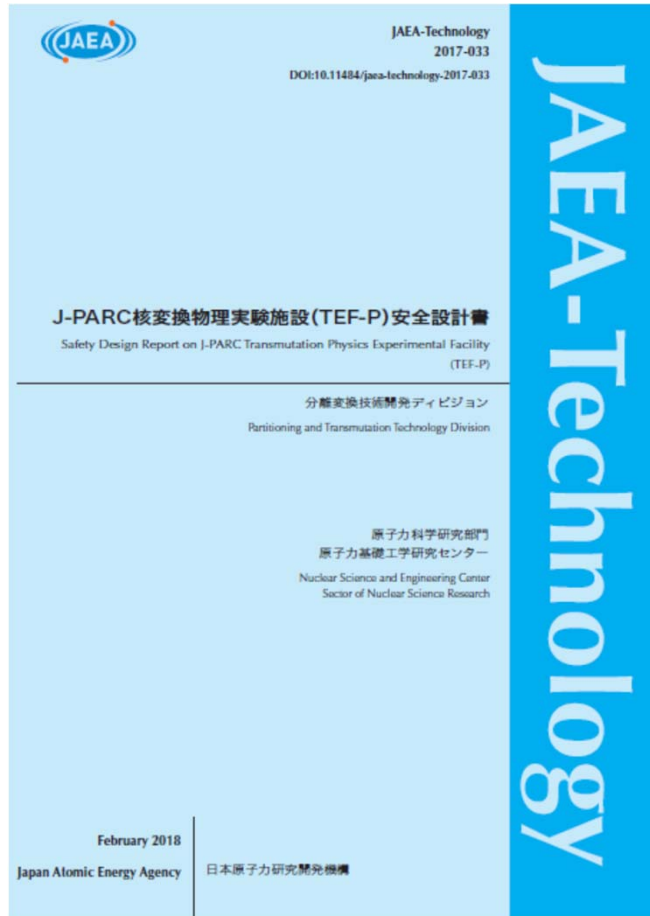


Maximum thermal power : 500W



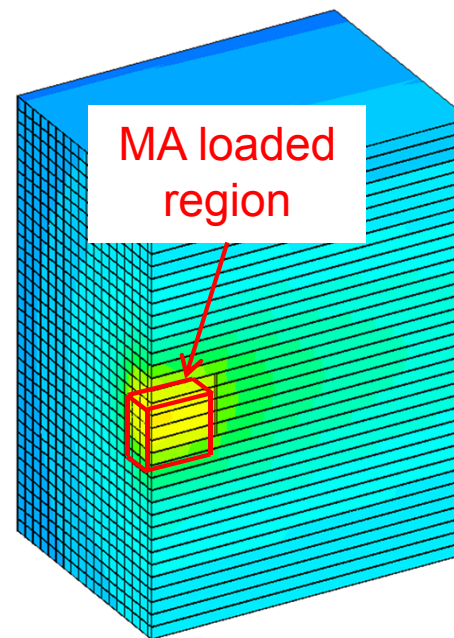
Safety Design Report of TEF-P

Safety design report of TEF-P was issued.



JAEA-Technology 2017-033
(sorry, in Japanese)

An example of safety analysis



Temperature
distribution of
TEF-P without
Cooling

Maximum temp.
303°C

< Melting Point of Pb

Radioactive exposure to circumference public :
< 5 mSv even without *Stop, Cool, Confine*

TEF-P facility was classified as Class B

Concluding Remarks



- ◆ Management of radioactive waste is one of the most challenging issues in the utilization of nuclear power.
- ◆ JAEA puts its priority of R&D on the **transmutation of MAs**, but **management of Pu** also becomes a very important issue, considering the current status of FR development.
- ◆ **Transmutation and utilization of FP** can change the paradigm; **“From waste to resource”**, if the ImPACT Program is successfully realized.
- ◆ **Synergy among various fields of science and technology, and also among countries**, is of great importance to tackle this issue.
 - Accelerator
 - Material
 - Separation
 - Remote handling, etc.